



Monte Carlo Particle Transport Methods

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Tavora, Pedro Vaz**



Monte Carlo Particle Transport Methods:

Monte Carlo Particle Transport Methods I. Lux, 2018-05-04 With this book we try to reach several more or less unattainable goals namely To compromise in a single book all the most important achievements of Monte Carlo calculations for solving neutron and photon transport problems To present a book which discusses the same topics in the three levels known from the literature and gives us useful information for both beginners and experienced readers It lists both well established old techniques and also newest findings *Monte Carlo Methods for Particle Transport* Alireza

Haghighat, 2020-08-09 Fully updated with the latest developments in the eigenvalue Monte Carlo calculations and automatic variance reduction techniques and containing an entirely new chapter on fission matrix and alternative hybrid techniques This second edition explores the uses of the Monte Carlo method for real world applications explaining its concepts and limitations Featuring illustrative examples mathematical derivations computer algorithms and homework problems it is an ideal textbook and practical guide for nuclear engineers and scientists looking into the applications of the Monte Carlo method in addition to students in physics and engineering and those engaged in the advancement of the Monte Carlo methods Describes general and particle transport specific automated variance reduction techniques Presents Monte Carlo particle transport eigenvalue issues and methodologies to address these issues Presents detailed derivation of existing and advanced formulations and algorithms with real world examples from the author's research activities **Acceleration**

Methods for Monte Carlo Particle Transport Simulations Lulu Li (Ph. D.), 2017 Performing nuclear reactor core physics analysis is a crucial step in the process of both designing and understanding nuclear power reactors Advancements in the nuclear industry demand more accurate and detailed results from reactor analysis Monte Carlo MC eigenvalue neutron transport methods are uniquely qualified to provide these results due to their accurate treatment of space angle and energy dependencies of neutron distributions Monte Carlo eigenvalue simulations are however challenging because they must resolve the fission source distribution and accumulate sufficient tally statistics resulting in prohibitive run times This thesis proposes the Low Order Operator LOO acceleration method to reduce the run time challenge and provides analyses to support its use for full scale reactor simulations LOO is implemented in the continuous energy Monte Carlo code OpenMC and tested in 2D PWR benchmarks The Low Order Operator LOO acceleration method is a deterministic transport method based on the Method of Characteristics Similar to Coarse Mesh Finite Difference CMFD the other acceleration method evaluated in this thesis LOO parameters are constructed from Monte Carlo tallies The solutions to the LOO equations are then used to update Monte Carlo fission sources This thesis deploys independent simulations to rigorously assess LOO CMFD and unaccelerated Monte Carlo simulating up to a quarter of a trillion neutron histories for each simulation Analysis and performance models are developed to address two aspects of the Monte Carlo run time challenge First this thesis demonstrates that acceleration methods can reduce the vast number of neutron histories required to converge the fission

source distribution before tallies can be accumulated Second the slow convergence of tally statistics is improved with the acceleration methods for the earlier active cycles A theoretical model is developed to explain the observed behaviors and predict convergence rates Finally numerical results and theoretical models shed light on the selection of optimal simulation parameters such that a desired statistical uncertainty can be achieved with minimum neutron histories This thesis demonstrates that the conventional wisdom e g maximizing the number of cycles rather than the number of neutrons per cycle in performing unaccelerated MC simulations can be improved simply by using more optimal parameters LOO acceleration provides reduction of a factor of at least 2 2 in neutron histories compared to the unaccelerated Monte Carlo scheme and the CPU time and memory overhead associated with LOO are small

Particle Transport Simulation with the Monte Carlo Method Leland Lavele Carter,E. D. Cashwell,1975 Monte Carlo Particle Transport H. Scott,N.

Gentile,R. Procassini,2005 Monte Carlo methods are frequently used for neutron and radiation transport These methods have several advantages such as relative ease of programming and dealing with complex meshes Disadvantages include long run times and statistical noise Monte Carlo photon transport calculations also often suffer from inaccuracies in matter temperature due to the lack of implicitness In this paper we discuss the Monte Carlo algorithm as it is applied to neutron and photon transport detail the differences between neutron and photon Monte Carlo and give an overview of the ways the numerical method has been modified to deal with issues that arise in photon Monte Carlo simulations Heat Conduction

Vyacheslav Vikhrenko,2011-11-30 The content of this book covers several up to date approaches in the heat conduction theory such as inverse heat conduction problems non linear and non classic heat conduction equations coupled thermal and electromagnetic or mechanical effects and numerical methods for solving heat conduction equations as well The book is comprised of 14 chapters divided into four sections In the first section inverse heat conduction problems are discuss The first two chapters of the second section are devoted to construction of analytical solutions of nonlinear heat conduction problems In the last two chapters of this section wavelike solutions are attained The third section is devoted to combined effects of heat conduction and electromagnetic interactions in plasmas or in pyroelectric material elastic deformations and hydrodynamics Two chapters in the last section are dedicated to numerical methods for solving heat conduction problems

Particle-transport Simulation with the Monte Carlo Method ,1975 Attention is focused on the application of the Monte Carlo method to particle transport problems with emphasis on neutron and photon transport Topics covered include sampling methods mathematical prescriptions for simulating particle transport mechanics of simulating particle transport neutron transport and photon transport A literature survey of 204 references is included GMT **Handbook of Nuclear Engineering** D. G. Cacuci,2010-09-14 This is an authoritative compilation of information regarding methods and data used in all phases of nuclear engineering Addressing nuclear engineers and scientists at all levels this book provides a condensed reference on nuclear engineering since 1958 *Advanced Monte Carlo for Radiation Physics, Particle Transport Simulation*

and Applications Andreas Kling, Fernando J.C. Barao, Masayuki Nakagawa, Luis Tavora, Pedro Vaz, 2014-02-22 This book focusses on the state of the art of Monte Carlo methods in radiation physics and particle transport simulation and applications the latter involving in particular the use and development of electron gamma neutron gamma and hadronic codes Besides the basic theory and the methods employed special attention is paid to algorithm development for modeling and the analysis of experiments and measurements in a variety of fields ranging from particle to medical physics

Development of a Multiple Perturbation Monte Carlo Method for Eigenvalue Problems and Implementation on Parallel Processors Amitava Majumdar, 1996 Particle-transport simulation with the Monte Carlo method Leland L. Carter, 1975

Nuclear Science Abstracts, 1976 *Nuclear Energy* Nicholas Tsoulfanidis, 2012-12-12 Nuclear Energy provides an authoritative reference on all aspects of the nuclear industry from fundamental reactor physics calculations to reactor design nuclear fuel resources nuclear fuel cycle radiation detection and protection and nuclear power economics Featuring 19 peer reviewed entries by recognized authorities in the field this book provides comprehensive streamlined coverage of fundamentals current areas of research and goals for the future The chapters will appeal to undergraduate and graduate students researchers and energy industry experts **Variance Reduction Techniques for the Monte Carlo**

Calculations of Electron Dose Distributions Mark Albert Holmes, 1995 Mathematics and Computations, Reactor Physics, and Environmental Analyses, 1995 **A Variationally-based Variance Reduction Method for Monte Carlo**

Particle Transport Problems Carla Lynn Barrett, 1999 **Domain Decomposition for Monte Carlo Particle Transport Simulations of Nuclear Reactors** Nicholas Edward Horelik, 2015 Monte Carlo MC neutral particle transport methods have long been considered the gold standard for nuclear simulations but high computational cost has limited their use significantly However as we move towards higher fidelity nuclear reactor analyses the method has become competitive with traditional deterministic transport algorithms for the same level of accuracy especially considering the inherent parallelism of the method and the ever increasing concurrency of modern high performance computers Yet before such analysis can be practical several algorithmic challenges must be addressed particularly in regards to the memory requirements of the method In this thesis a robust domain decomposition algorithm is proposed to alleviate this along with models and analysis to support its use for full scale reactor analysis Algorithms were implemented in the full physics Monte Carlo code OpenMC and tested for a highly detailed PWR benchmark BEAVRS The proposed domain decomposition implementation incorporates efficient algorithms for scalable inter domain particle communication in a manner that is reproducible with any pseudo random number seed Algorithms are also proposed to scalably manage material and tally data with on the fly allocation during simulation along with numerous optimizations required for scalability as the domain mesh is refined and divided among thousands of compute processes The algorithms were tested on two supercomputers namely the Mira Blue Gene Q and the Titan XK7 demonstrating good performance with realistic tallies and materials requiring over a terabyte of aggregate

memory Performance models were also developed to more accurately predict the network and load imbalance penalties that arise from communicating particles between distributed compute nodes tracking different spatial domains These were evaluated using machine properties and tallied particle movement characteristics and empirically validated with observed timing results from the new implementation Network penalties were shown to be almost negligible with per process particle counts as low as 1000 and load imbalance penalties higher than a factor of four were not observed or predicted for finer domain meshes relevant to reactor analysis Load balancing strategies were also explored and intra domain replication was shown to be very effective at improving parallel efficiencies without adding significant complexity to the algorithm or burden to the user Performance of the strategy was quantified with a performance model and shown to agree well with observed timings Imbalances were shown to be almost completely removed for the finest domain meshes Finally full core studies were carried out to demonstrate the efficacy of domain decomposed Monte Carlo in tackling the full scope of the problem A detailed mesh required for a robust depletion treatment was used and good performance was demonstrated for depletion tallies with 206 nuclides The largest runs scored six reaction rates for each nuclide in 51M regions for a total aggregate memory requirement of 1.4TB and particle tracking rates were consistent with those observed for smaller non domain decomposed runs with equivalent tally complexity These types of runs were previously not achievable with traditional Monte Carlo methods and can be accomplished with domain decomposition with between 1.4x and 1.75x overhead with simple load balancing

Neutronics Jean-François Parisot, 2015 Neutronics or neutron physics is the study of neutrinos travelling through matter of conditions for a chain reaction and of changes in matter's composition due to nuclear reactions It makes it possible to design and operate nuclear reactors and fuel cycle facilities Publisher **INIS Atomindex**, 1985 An

Introductory Guide to EC Competition Law and Practice Valentine Korah, 1994

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